



ATR National Scientific User Facility Draft Users Week 2010 Schedule June 7-11, 2010 Idaho Falls, ID

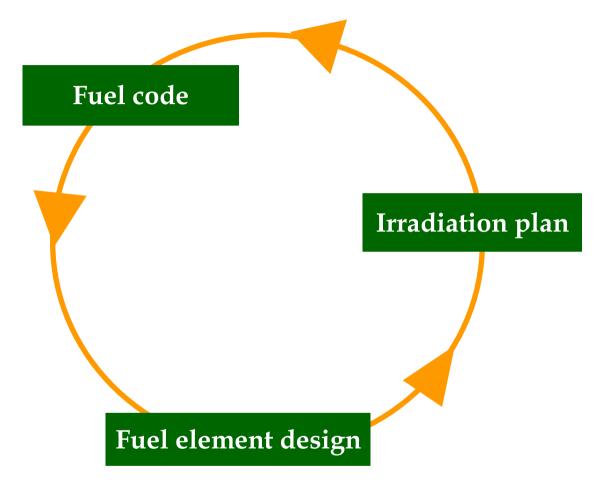
Fuel design, irradiation programme and modelling, Application to several fuels for GENIV systems.

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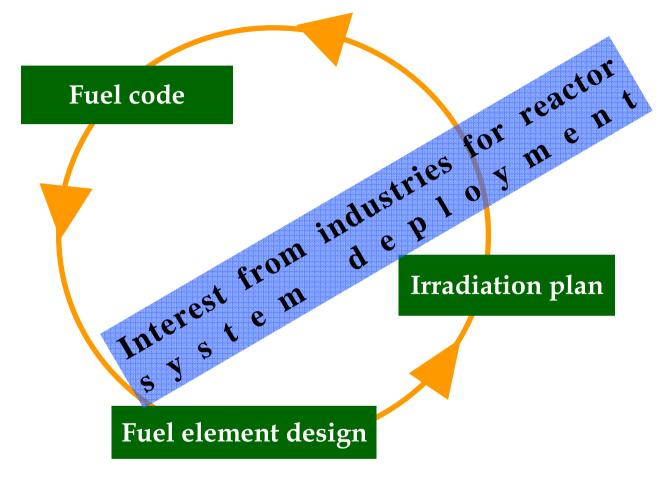
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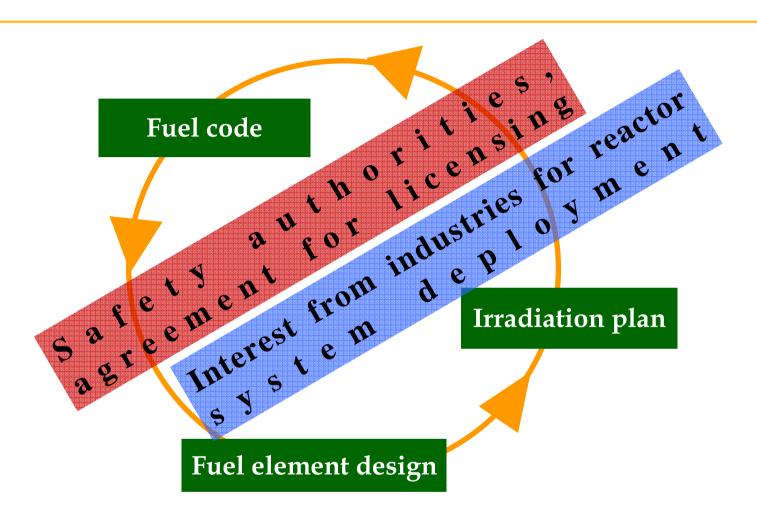






Lower margin for fuel design → performances increase





Lower margin for fuel design → performances increase

Less uncertainties and higher confidence → safety enhancement



DEFINITIONS FOR EVALUATION

Fuel design phases, Irradiations types, Fuel code levels

→ Choice of examples

PART 1: FUEL DESIGN AND QUALIFICATION WITH IRRADIATION PLAN

PART 2: FUEL CODE AND VALIDATION WITH IRRADIATION

PART 3: ILLUSTRATIONS OF IRRADIATION-FUEL DESIGN-MODELLING WITH GENIV FUEL EXAMPLES

CONCLUSION



Definition of fuel design



- 4 phases:
 - Selection
 - Prospective approach to demonstrate feasability, to assess materials & design options
 - Development
 - Identify limitations and all items of interest for R&D
 - Optimisation
 - Improvement of safety and performances with : fabrication process, materials optimisation, design ,
 - Normal and off-normal conditions
 - Qualification
 - Full size demonstration under prototypic conditions : neutronic + thermomechanic + thermohydraulic + thermodynamics
 - Licensing of fuel/core by regulator by identification of fuel limits

→ A quantitative and more detailed evaluation : Technical Readiness Level scale*

^{*} Global'09, K. A. McCarthy and K. O. Pasamehmetoglu, Paper 9477



Definition of Irradiation types in reactor



NORMAL CONDITIONS

- Analytical
 - Objective → 1 phenomenon (creep, swelling, gas diffusion....)
 - Irradiation design:
 - Geometry: conventional pellet or dedicated geometries (disk, samples, ...),
 - Conditions: most are fixed to test 1 parameter
 - Monitoring and in-situ measurements
 - Reactor:
 - → In MTR
 - + in-pile measurements (pressure, FG release, fuel temperature, fuel stack elongation, fuel pin outer diameter change) with possibility to fix some conditions
 - thermal spectrum (or screen), different limit conditions except in dedicated loop,
 - miniature fuel rodlet, limited irradiation time
 - → In prototype
 - + representative conditions including fast neutron dose and high BU
 - no instrumentation, « Cook and look »

Integral or semi-integral

- Objective → phenomena coupling (Fission gas release, temperature, clad strain, ...)
- Irradiation design:
 - Pin or shorted pin with representative radial geometry
- Reactor : same



Definition of Irradiation types in reactor



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 - Reactor:
 - → In MTR
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 - thermal spect
 - miniature fuel
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- Integral or semi-integral
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 - Irradiation design:
 - Pin or shorted pin with representative radial geometry
 - Reactor: same



ruel stack

uns except in dedicated loop,

... to set some conditions



OFF-NORMAL CONDITIONS

- Analytical
 - MTR: overpower test on re-fabricated irradiated pin (slow transient)
 - Furnace heating test in hot lab: 1 pellet with/wo clad
 - annealing treatment or temp. ramp or even a local heating to have a temperature gradient on irradiated fuels above 2500℃ with different atmospheres (MERARG II – DURANCE devices in LECA hot cells facility at CEA-Cadarache)
 - on-line measurement of gas release : FG and volatile FP release.
- Integral or semi-integral -> phenomena coupling
 - Hot cells : severe accident conditions on irradiated rodlet (VERDON in LECA)
 - Safety reactor for fast transient test on a full size pin (CABRI, TREAT, ...) or severe accident on pin bundle (*PHEBUS*).



Definition of code grade levels



- LEVEL 1; Pre-design code
 - Empirical models, simple ones and limited coupling
 - Check behaviour for pre-designing fuel
- LEVEL 2; R & D code
 - Dedicated models and material laws with models coupling
 - For fuel design & experimental irradiation design & post irradiation calculation
- LEVEL 3; Fuel performance code
 - 1.5 D (axi-symmetry), 2 D or 3D thermo-mechanical analysis,
 - Modelling increasingly mechanistic and multi-scale approach
 - Validation with large experimental database
- LEVEL 4; Predictive code « THE HOLY GRAIL !! »

 That can be used outside of its validation area with a high confidence on results:
 - Reliability (physics based models and model coupling)
 - Availability (application on a large area with spread irradiation database and material database)
 - Accuracy (high level for all situations)
 - Multiscale modelling (bubbles, grain, pellet, fuel element level) for whole fuel element evaluation as well as local effects (1.5D-2D-3D) and non symetric effects
 - with microstructure and irradiation effects.
 - Coupling between thermochemistry, thermodynamic, transport theory, neutronic and thermal/mechanical conventional analysis



CHOICE OF EXAMPLES



- GFR, plate and pin fuel with mixed carbide and refractory cladding (composite)
 - Selection phase for fuel design
 - Fuel code development before irradiation programme
 - · Irradiations for feasability studies on fabrication and behaviour
- HTR, particle fuel
 - Optimisation-qualification phase,
 - Fuel code development before the irradiation programme
 - Irradiations able to evaluate and improve both behaviour (models) and fuel fabrication : analytical and integral irradiations
- SFR, driver MOX fuel
 - Qualification phase
 - Fuel code existing, models improvement on-going
 - · Irradiations to extend experimental database
- SFR, transmutation homogeneous MOX fuels (U_{0.78}, Pu_{0.2}, Am_{0.013}, Np_{0.007}, Cm_{0.006})O_{2-x}
 - Development phase
 - Fuel code existing, adaptation of some models
 - Irradiations to build dedicated database
- SFR, MABB minor actinide bearing blanket (U_{0.8}, Am_{0.153}, Np_{0.034}, Cm_{0.013})O_{2-x} or (U_{0.8}, Am_{0.2})O_{2-x}
 - Selection-Development phase
 - Fuel code existing, need of dedicated models + adaptation of existing models
 - Irradiations to demonstrate limits and for model validation

Others possible applications: dispersion fuels for transmutation, metal fuel,





• PART I: FUEL DESIGN AND QUALIFICATION WITH IRRADIATION PLAN

SELECTION PHASE



• Objective:

- Based on a first fuel design with several options (design/material)
- → Evaluate performances towards requirements

Irradiation

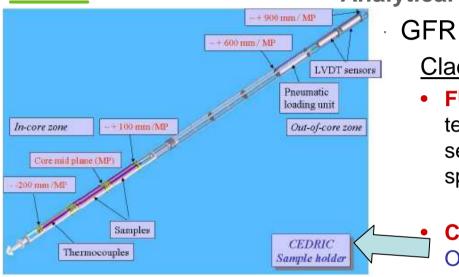
 Screening irradiations in order to remove options due to critical points



SELECTION PHASE : example











Cladding:

• FUTURIX-MI (select the best cladding for high temp.-1000℃ and fast fluence-40 dpa), PHENIX, self-heating device with sample holder, different specimens, temperature evaluation by monitors.

CEDRIC (SiC creep)

OSIRIS, SiC fiber under constant stress (200MPa) and LVDT for on-line elongation measurement.



Concept:

 FUTURIX-Concept (select the best concept for GFR conditions, thermomechanical & thermochemical)

PHENIX, pin-std rig with several fuel types (coated particles, honeycomb structure, nitride, carbide fuels,...).



DEVELOPMENT PHASE



Objective:

- → Reference design
- → Specifications fulfill-Performances-limits-critical points

Irradiation

- Prototypic fuel (lab. scale fabrication)
- Intermediate conditions (Burn Up and dose / with a step by step approach)
- Identification of normal conditions life limiting phenomena



DEVELOPMENT PHASE: example



- Analytical
 - Homogeneous transmutation of Am
 - AM1 in JOYO reactor for BOL phenomena
 - GFR
 - UPuC:
 - GOCAR (effect of temperature on MC swelling)
 SILOE reactor, special design with gap adjustment for temp. control, thermocouple
- Integral
 - Homogeneous transmutation of Am and Np
 - **SUPERFACT** (behaviour at intermediate burn-up) PHENIX, mean LHR, 7at%
 - GFR
 - UPuC:
 - NILOC, HFR
 - **NIMPHE**, PHENIX
 - L414, JOYO
 - AC3, FFTF
 - Transient tests in TREAT....
 - Plate and pin prototype first test



DEVELOPMENT PHASE: example

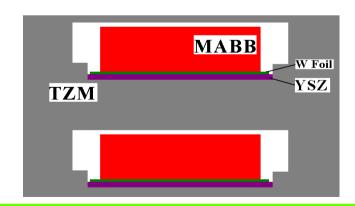


Analytical

- MABB
 - MARIOS & DIAMINO in HFR & OSIRIS
 - Screening experiment on the impact of MABB fabricated microstructure on He behaviour under irradiation at different temp.



- Temperature <u>controlled</u> with gas composition and <u>constant</u> in the disk with axial thermal exchanges to avoid thermal gradients
- Radial and axial gaps to let free swelling of discs.
- Conditions and parameters:
 - 4 temp. : 600-800-1000-1200℃
 - 2 microstructures
 - 2 He production rates
- Irradiation: 2010 (MARIOS) 2011 (DIAMINO).





OPTIMISATION PHASE



• Objective:

- → Performance improvement
 - → Reliability
 - → Safety
 - →Cost
 - → Limits

Irradiation

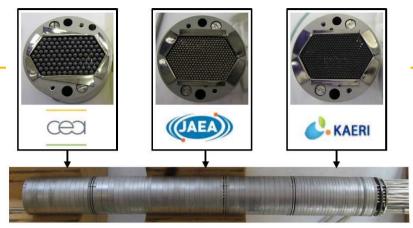
- Representative conditions: in FR or in dedicated loop system in MTR
- Representative fuel element with industrial fabrication

OPTIMISATION PHASE: example



- **Analytical**
 - HTR
 - PYCASSO in HFR

Objectives (CEA):



- PyC densification kinetics (tangential and radial strain) under neutron fluence
- Comparing geometrical changes with ATLAS numerical calculations

Design

- Several conditions: free of stress or representative stress on PyC
- Two stacks of 14 disks are irradiated, at 900 & 1100℃, with maximum fast neutron fluence 2 x 10²⁵ m⁻²
- Temperature monitoring with gas composition adjustment + TC on device structure
- Avril. 2008 Avril 2009

Schematic representation of the CEA specimens for PYCASSO-I.



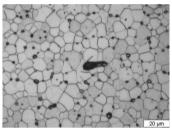
	kernel/buffer	kernel/buffer/PyC	kernel/buffer/SiC	kernel/buffer/SiC/PyC
Ø/épaisseurs [µm]	1000/250	1000/250/40	1000/215/35	1000/215/35/40
échelles schémas : x 10 et taille réelle sous format A4				

OPTIMISATION PHASE: example

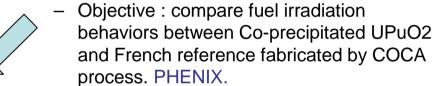


Analytical

- Homogeneous transmutation
 - AF-2C, 2D, ATR
 - FCCI at high Burn Up
- Integral
 - SFR : coprecipitated MOX

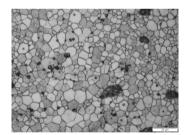








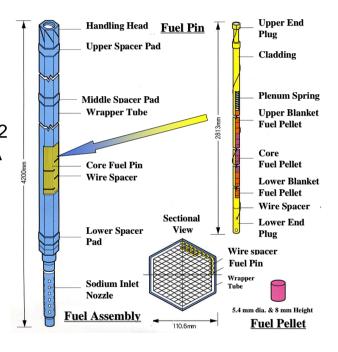
- 2 O/M (1,937 and 1,965),
- 2 pins with 2 fabrication routes (direct or dilution) → 2 microstructures
- Sept. 2008 March 2009 , 4-5 at% -420W/cm



- Homogeneous transmutation
 - GACID step1&2
 - Objective: demonstration that MOX driver fuel can transmute MA's (Np/Am/Cm) with reduced impact on fuel behaviour. JOYO & MONJU
 - Design & fabrication process in progress

GACID

 $\frac{\text{STEP1}}{\text{STEP2}}: (\text{U}, \text{Pu}_{0,25}, ^{241}\text{Am}_{0,03}, \text{Np}_{0,01})\text{O}_{\text{x}} \\ \underline{\text{STEP2}}: (\text{U}, \text{Pu}_{0,25}, ^{241}\text{Am}_{0,03}, \text{Np}_{0,01}, \text{Cm}_{0,002})\text{O}_{\text{x}}$





QUALIFICATION PHASE



Objective:

- Demonstration of performance and reliability in normal and off-normal conditions at full sub-assembly scale
- Directly useful for industrial applications : fuel cycle (specification fulfill) and reactor prototype (safety licensing).

Irradiation

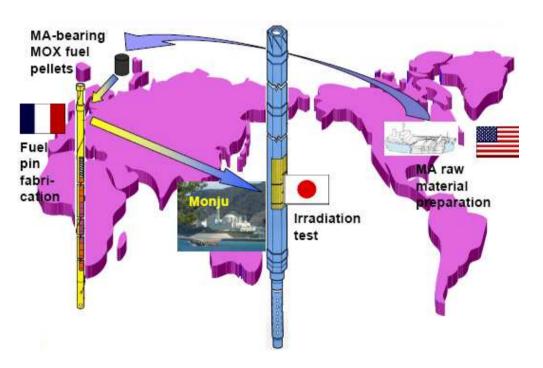
- Full sub-assembly scale
- Fabrication in an industrial plant with pilot processes
- Representatives conditions for sub-assembly in a prototypic reactor



QUALIFICATION PHASE: example



- Integral
 - Homogeneous transmutation
 - GACID step 3
 - Bundle-scale MA-bearing Fuel Irradiation Demonstration in MONJU







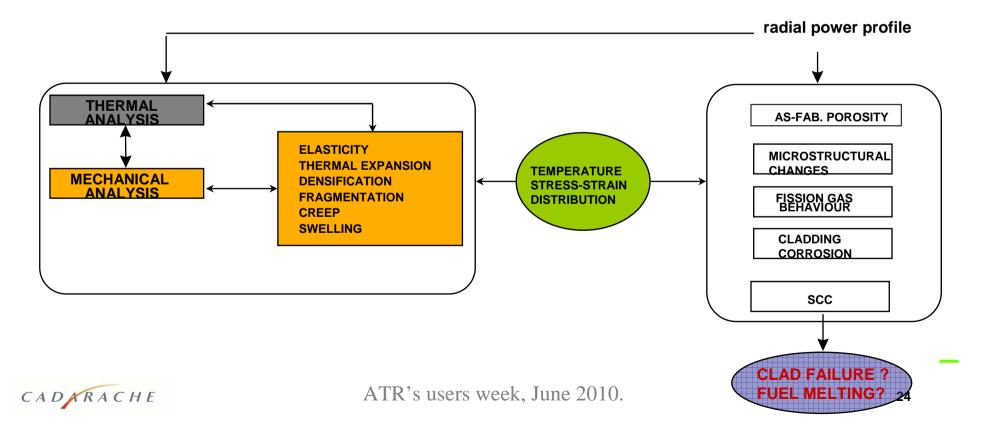
• PART II: MODELLING AND EXPERIMENTAL VALIDATION



Fuel Modelling: principles



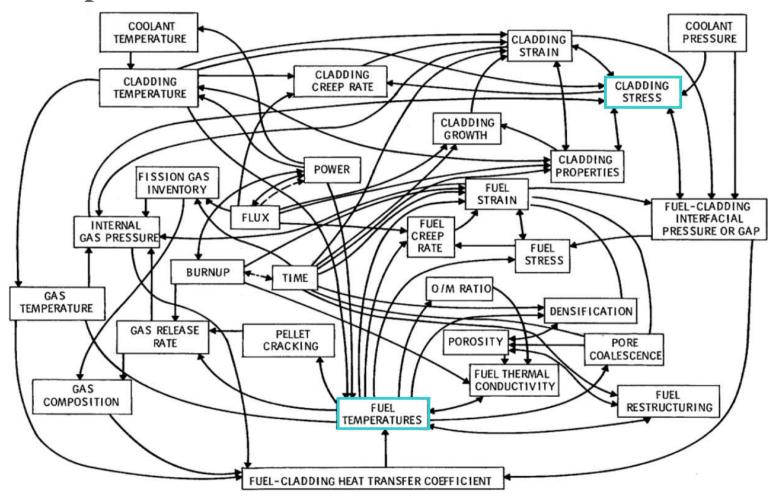
- In pile behaviour: several phenomena
 - Complex
 - Simultaneous
 - Coupled
 - ...and with irradiation induced changes in materials
- Schematic view of how models interact



Fuel Modelling: real life



•Complete view of how models interact !!!



•Ref. Manning et al., MMSNF conference



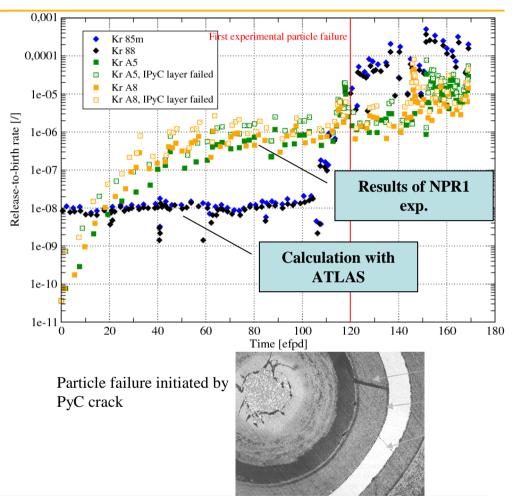
HTR fuel: PyC densification

Current model

- PyC is orthotropic and anisotropy evolves with conditions
- Stresses are relaxed in PyC by irradiation induced creep.
- Too high stresses could lead to PyC and SiC cracks.

Results : C/E

- No direct measurement, only particle failure rate (Xe & Kr release to birth)
- Calc. surestimates particle failure

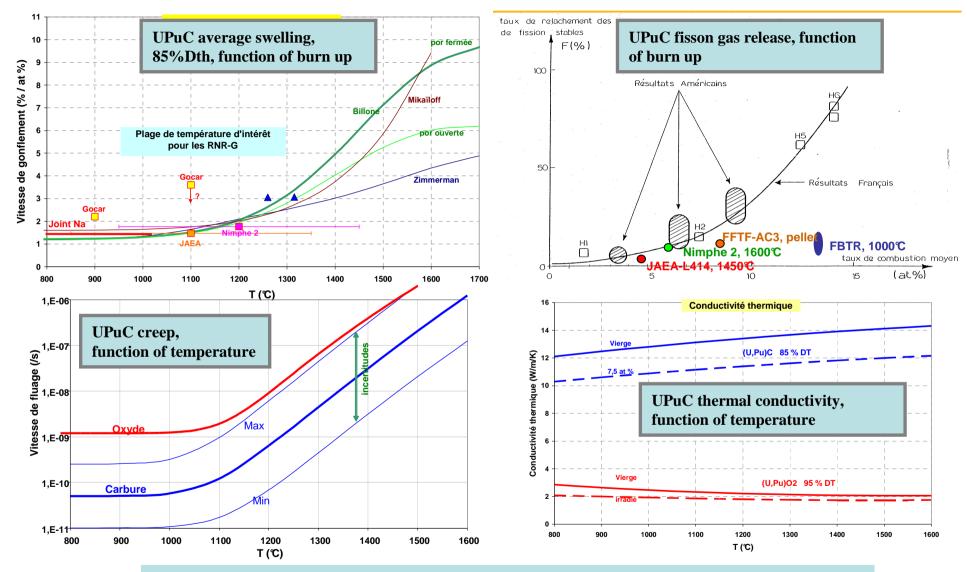


→Current C/E discrepancies have significant impact on :

Particle failure evaluation (SiC Weibull approach) → first HTR fuel criteria (direct cycle)



GFR fuel: UPuC models



→ UPuC behaviour very complex with high level of uncertainties



SFR MOX fuel: microstructure evolution

Zone where radial cracks heal Circumferential crack Radial cracks Radial cracks New cracks probably formed during

Current Models

Different conditions, space and time dependent: temp. and temp. gradient, LHR evolution, local composition, evolution (O, actinides)

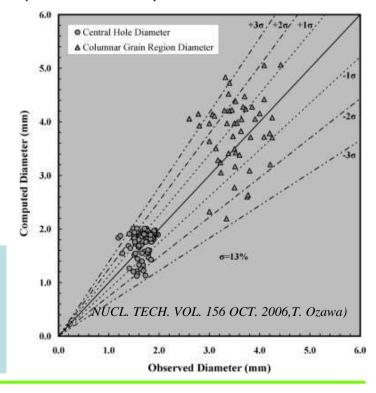
- Gas and solid swelling
- Porosity movement
- Coalescence model (central hole)
- Crack location

- Results: E/C
 - Exp. parameters : central hole, columnar grain diameter, clad strain, porosity field,
 - Calc. predict central hole and columnar grain at <u>+</u>25%

→Current C/E discrepancies have significant impact on:

Margin to melt

Max BU depends on central hole closure to avoid severe FCMI.





SFR MOX fuel: Fission gas release

Current model

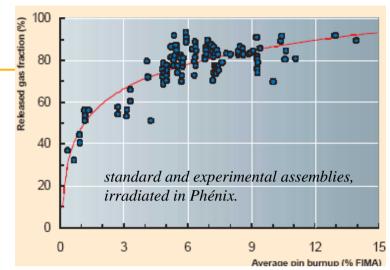
depending on almost all the others models (JOG, FCCI, swelling, clad creep, restructuration, diffusion...)

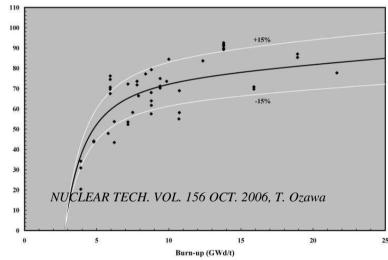
3 types of model:

- Simple correlation with temp., BU
- Grain size (Booth-like model)
- Full model including known mechanisms (defect, intra-extra grain diffusion, cavities type, bubbles)

Results : C/E

- Exp. parameters : FGR measurement or in-situ pressure and FP analysis + fuel retained gas distribution
- Calc. predict FGR at ±15%





→ Current C/E discrepancies have significant impact on :

Strong impact on temperature

As inner pressure is a clad limiting point, conservative approach



SFR MOX fuel: Pellet to clad join (JOG)

Current model

Cs production → Chemical state prediction (Cs₂MoO₄) → radial (pellet) and axial (gap) transport

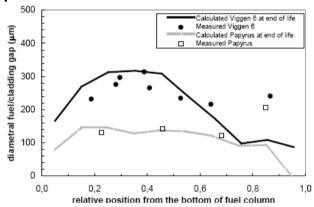
Results : C/E

Exp. parameter : JOG thickness, retained Cs

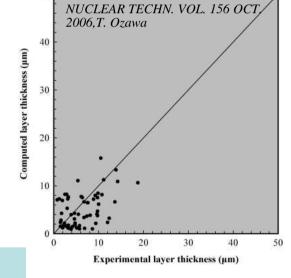
Cal. underestimate JOG thickness at TOP and even at PPN

→high uncertainties on transport mechanism, axial extrusion,

temperature



Comparison cal/meas for axial evolution of fuel clad gap with GERMINAL Code



→ Current C/E discrepancies have significant impact on :

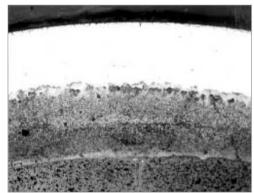
temperature and clad strain (max. burn up)



JOG

Mo

SFR MOX fuel: FCCI



Extensive cladding internal corrosion (~40% initial thickness), in an experimental Phénix fuel element, 15–15 Tiɛ steel, 16.9at%, 155 dpa.

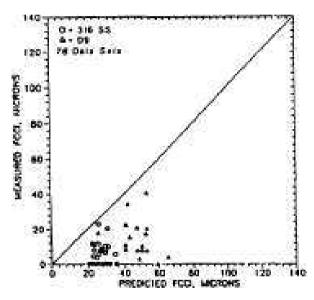
Current Models

good understanding of mechanism but full model (neutronic-thermodynamic-transport-thermodynamic) is still missing

Simple correlation with BU, temperature



- Exp. parameters : axial internal corrosion depth
- Calc. surestimates FCCI in all cases



measured maximum FCCI depths in FFTF with predicted value (IAEA, TECDOC-1083)

→ Current C/E discrepancies have significant impact on :

Clad failure risk evaluation



SFR MOX fuel: Thermal behaviour



- Challenge: reduce uncertainties to increase temperature (B.U./LHR) with a reliable approach
- Objective: accurate prediction of temperatures (+/- 50-100℃) everywhere in fuel pin & in any situation.
- Open questions:
 - Heat transfer in the gap before full gap closure with JOG?
 - Heat transfer at high burnup (JOG conductivity)?
 - JOG axial transfert, impact on temperature at top of FC?
 - Thermal conductivity λ of fuel at high burn-up (large degradation? effect of species redistribution?)

→In reactor measurement of centre-line temperature

Approach currently chosen to validate our thermal calculation Need to be associated with others measurements to fix the others parameters (FGR, diameter change,...)



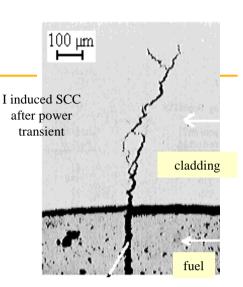
SFR MOX fuel: Mechanical behaviour

Challenge :

- reduce uncertainties to increase burnup/LHR
- improve knowledge to propose enhancement (microstructure or dopant) like PWR

Objectives

- Predict all dimensional changes: clad strain (5-10% of max. strain), gap closure
- Predict the risk of clad failure during power increase



Work-hardened 316 Ti (1981) Work-hardened 316 Ti (1981) Work-hardened 316 Ti (1981) Work-hardened 15-15 Ti (1985) EM12 (1990) 800 1,000 1,200 1,400 1,600 1,800 Elevation (mm from bottom of pin)

Open questions

- Fuel: swelling, creep, cracking coupling?
- Clad: Creep, swelling at high burn-up/temperature (ODS)?
- FCCI: high accuracy for all clad and all fuel compositions and conditions (model based on FP diffusion, oxide compounds thermodynamic). How clad properties affected by FCCI?
- FCMI: max burn up for normal conditions and threshold of over-power or over-temperature during transient?
- what happen with MC/ODS, dispersed fuel/SS, MC/SiCSiC_f (balance fuel creep-swelling and clad creep-swelling)?
- → Mechanical properties of fresh and irradiated fuel and cladding
- → In reactor : clad strain measurement and ramp tests in MTR



Modelling challenges



GFR fuel

- MC creep, swelling, FGR comprehensive models
- SiC-SiC_f yield and fracture strength, swelling vs temperature, fluence
- MC-SiCSiC_f coupling: clad stress and max strain at high doses

HTR

Models validation with experimental results (characterisation based)

SFR MOX driver fuel

- Pellet behaviour at high BU and effect on clad stress
- Burn up linked phenomena: FCCI (clad wastage), JOG
- During transient: thermomechanical behaviour at high burn up
- Species diffusion coupled with thermodynamic

MOX Homogeneous fuel

 Impact of MA on FCCI, FCMI at high burn up (thermodynamic) and margin to melt to be checked

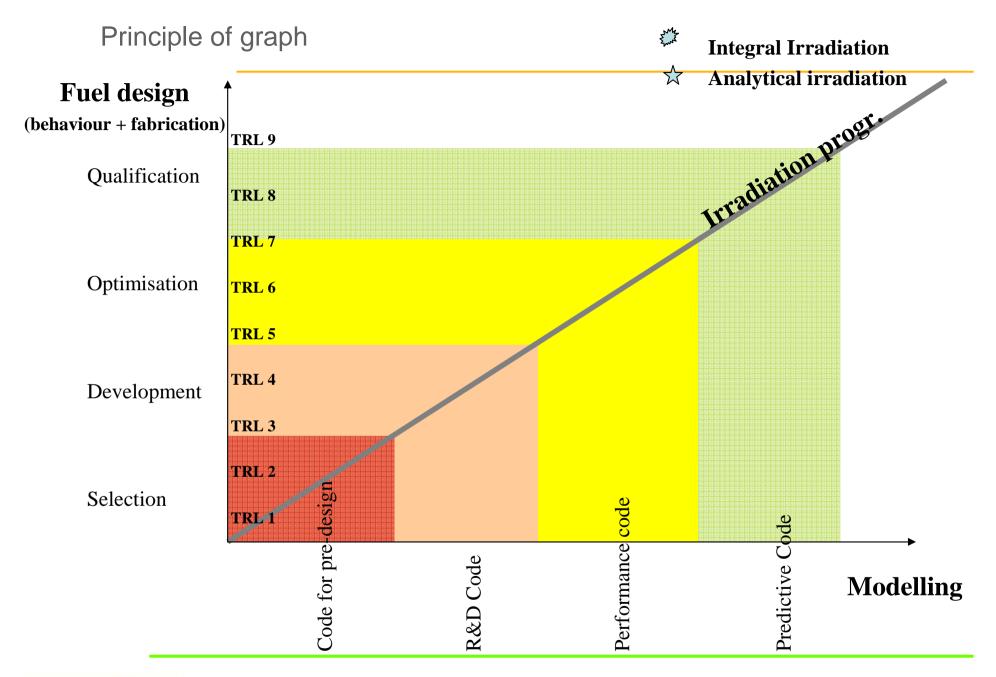
MABB

 Development of models based on UO₂ behaviour, completed with Helium production and release. Need to take microstructure (homogeneous/heterogeneous) into account.

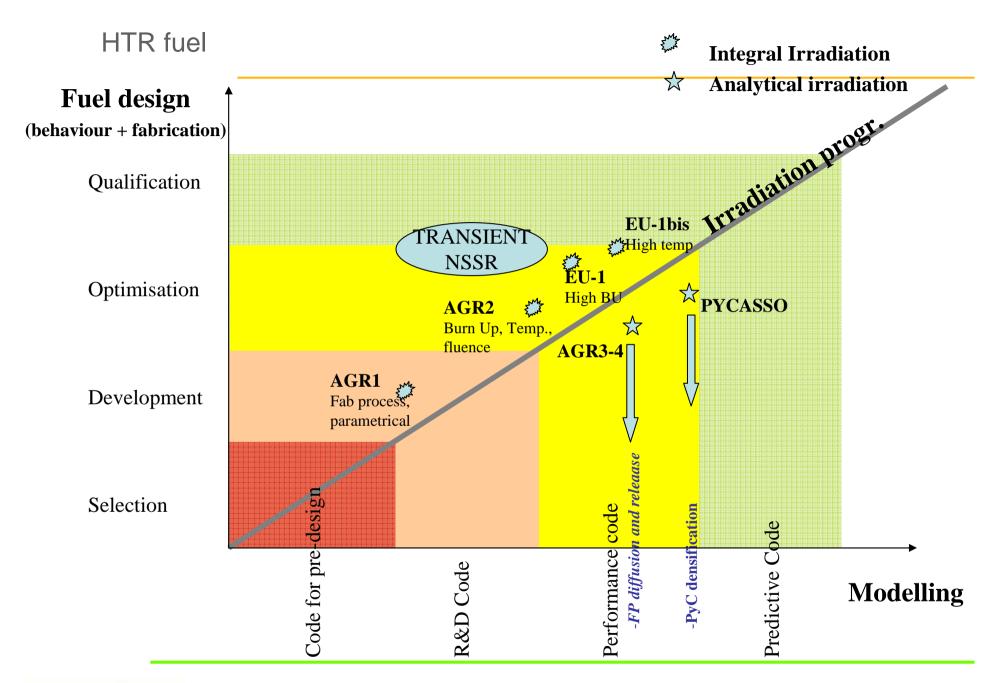




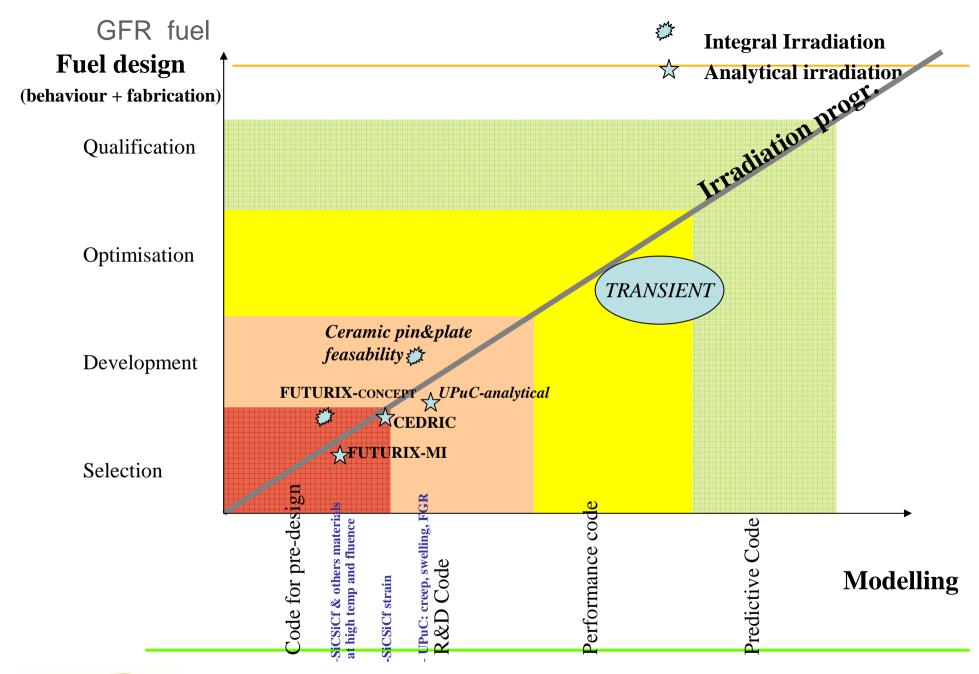
 PART III: ILLUSTRATIONS OF IRRADIATION-FUEL DESIGN-MODELLING WITH GENIV FUEL EXAMPLES





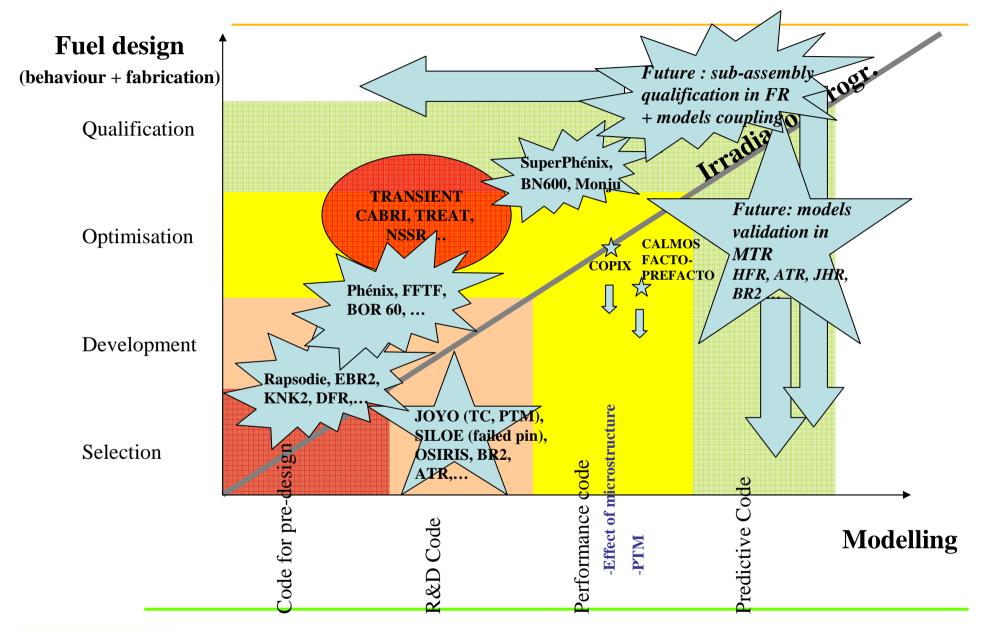




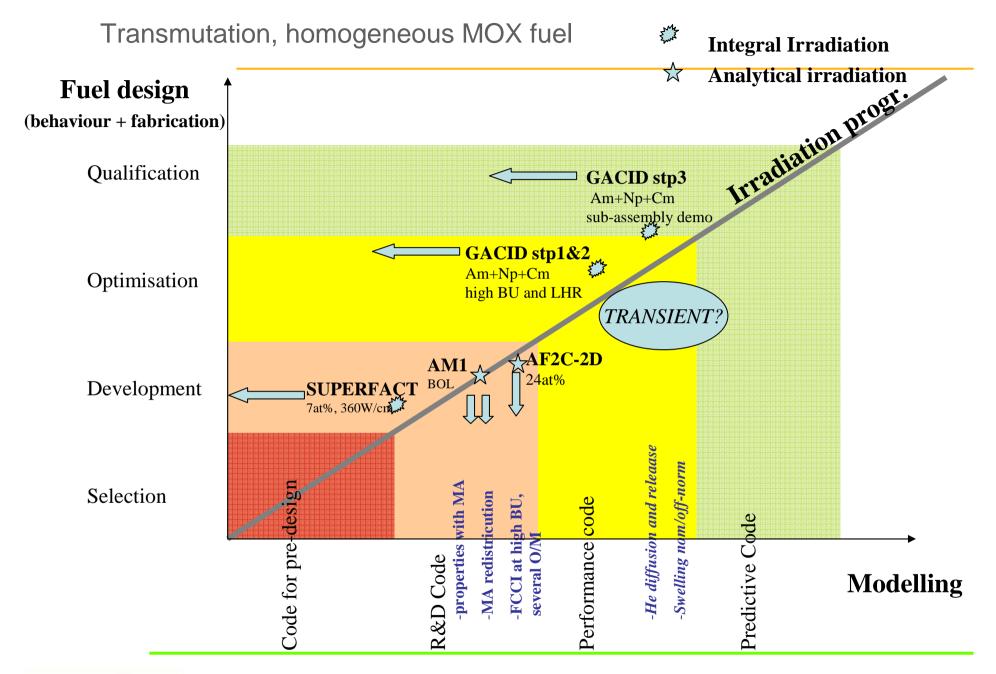




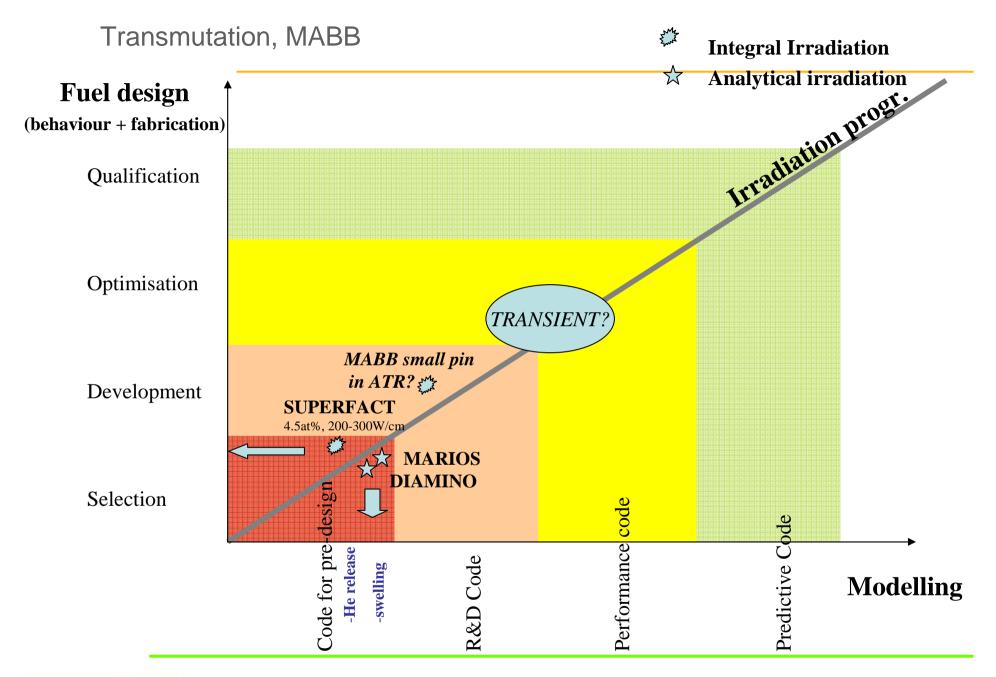
SFR- MOX driver fuel















Classical link between fuel design-irradiation-code

Irradiation gives results for code qualification

Fuel design → irradiation design → Irradiation results → Models devt. and code validation

Since 2000....

- → Because there is less and less MTRs and SFRs where it's possible to make experiments
- → Because of cost : impossible to have several hundreds points for 1 configuration
- → Because modelling has undertaken major steps (numerical methods, computer capacity, fuel behaviour knowledge, ...)

Models validation gives specification for irradiations

Fuel design \rightarrow calculation \rightarrow models requirements \rightarrow irradiation design \rightarrow Irradiation results



Nevertheless,

- Less than <u>20 parameters</u> may be checked with PIE results
- Less than <u>5 parameters</u> may be measured in pile
- Part of calculation uncertainties are due to data evaluation (fabrication- properties-irradiation conditions)

Improvement on modelling MUST BE COMPLETED with

- → More accuracy on fabrication data and irradiation conditions
- → More instrumentation in core, especially in MTR (also in prototype?)
- → More characterisations in hot cell.





THANK YOU FOR YOUR ATTENTION THANK YOU FOR INVITING ME HERE

